Purdue University School of Nuclear Engineering

NUCL 552 T TH

Instructor:

M. Ishii

Office: NUCL 112C Tel: 494-4587

Office Hours: T Th

Text Book:

N. Todreas & M. Kazimi, Nuclear Systems I

M. Ishii and T. Hibiki, Thermo-fluid Dynamics of Two-phase Flow

References:

E. E. Lewis, Nuclear Power Reactor Safety

R. T. Lahey & J. J. Moody, The Thermal Hydraulics of Boiling Water

Nuclear Reactor

L. S. Tong & J. Weisman, Thermal Analysis of Pressurized Water

Reactors

Course Content:

1. Neutron Chain Reaction

- 2. Nuclear Reactor Systems
- 3. Safety Characteristics
- 4. Safety Assessment
- 5. Reactor Licensing
- 6. Introduction to Vector Calculus
- 7. Formulation of Thermal-Hydraulic Problems
- 8. Physical Laws and Balance Equations
- 9. Constitutive Relation and Closure Relation
- 10. Single Phase Thermal-Hydraulics
- 11. Fuel Thermal Transient and Melting
- 12. Two-Phase Flow Formulation
 - Homogeneous Flow
 - Drift Flux Model
 - Two-Fluid Model

13. Two-Phase Thermal-Hydraulics

Scaling, Critical Heat Flux, Flooding, Flow Instability, Flow Excursion, Post Dryout Phenomena, Critical Flow

14. Special Topics Important for Reactor Safety

Grading Basis (Tentative)

Homework 30% Midterm Exam 30% Final Exam 40%

Homework

Assignments will be due at the time stated when work is assigned, unless otherwise specified. Late homework will not be accepted for grade; however, all homeworks should be submitted before the end of the semester.

- Midterm and Final Exam will be closed book and notes.
- An A will be given if the total score is $\geq 85\%$. If the total grade is less than 50%, the grade will be F. The other grades will be determined after considering the class distribution.